



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
1600 E. LAMAR BLVD.
ARLINGTON, TX 76011-4511

March 25, 2014

EA-14-033

Jeremy Browning, Site Vice President
Arkansas Nuclear One
Entergy Operations, Inc.
1448 SR 333
Russellville, AR 72802-0967

SUBJECT: ARKANSAS NUCLEAR ONE, UNITS 1 AND 2 -NRC SPECIAL INSPECTION
REPORT 05000313/2013406 AND 05000368/2013406; PRELIMINARY
GREATER-THAN-GREEN FINDING

Dear Mr. Browning:

On February 27, 2014, the U.S. Nuclear Regulatory Commission (NRC) completed a reactive inspection pursuant to Inspection Procedures 93812, "Special Inspection," at your Arkansas Nuclear One, Units 1 and 2. The enclosed inspection report documents the inspection results which were discussed on February 27, 2014, with Mr. Dale James, Director, Regulatory and Performance Improvement, and other members of your staff.

The special inspection commenced on December 2, 2013, in accordance with NRC Management Directive 8.3, "NRC Incident Investigation Program," and Inspection Manual Chapter 0309, "Reactive Inspection Decision Basis for Reactors," based on the initial deterministic criteria evaluation made by the NRC on November 25, 2013. The special inspection reviewed the circumstances surrounding an issue which your staff identified on November 14, 2013, and examined activities conducted under your license as they relate to security and compliance with the Commission's rules and regulations and with the conditions of your license.

The enclosed inspection report discusses a finding that has preliminarily been determined to be a Greater-than-Green finding, of greater than very low security significance, that may result in the need for further evaluation to determine significance, and therefore the need for additional NRC action. This finding was assessed based on the best available information, using the Physical Protection Significance Determination Process. The deficiencies were promptly corrected or compensated for, and the plant was in compliance with applicable physical

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protection and security requirements within the scope of this inspection before the inspector arrived onsite. The basis for the NRC's preliminary significance determination is described in the enclosed report. The final resolution of this finding will be conveyed in separate correspondence.

The finding is also an apparent violation of NRC requirements and is being considered for escalated enforcement action in accordance with the Enforcement Policy, which can be found on the NRC's Web site at <http://www.nrc.gov/about-nrc/regulatory/enforcement/enforce-pol.html>.

In accordance with NRC Inspection Manual Chapter (IMC) 0609, we intend to complete our evaluation using the best available information and issue our final determination of safety significance within 90 days of the date of this letter. The significance determination process encourages an open dialogue between the NRC staff and the licensee; however, the dialogue should not impact the timeliness of the staff's final determination.

Before we make our final significance determination, we are providing you an opportunity to (1) attend a Regulatory Conference where you can present to the NRC your perspective on the facts and assumptions the NRC used to arrive at the finding and assess its significance, or (2) submit your position on the finding to the NRC in writing. If you request a Regulatory Conference, it should be held within 30 days of the receipt of this letter, and we encourage you to submit supporting documentation at least one week prior to the conference in an effort to make the conference more efficient and effective. In the interest of protecting sensitive, security-related information, if a Regulatory Conference is held, it will not be open for public observation. If you decide to submit only a written response, such submittal should be sent to the NRC within 30 days of your receipt of this letter. If you decline to request a Regulatory Conference or submit a written response, you relinquish your right to appeal the final SDP determination, in that by not doing either, you fail to meet the appeal requirements stated in the Prerequisite and Limitations sections of Attachment 2 of IMC 609.

Please contact Mr. Mark Haire, Chief, Plant Support Branch 1, Division of Reactor Safety, at (817) 200-1527 within 10 days of the date of this letter to notify the NRC of your intentions. If we have not heard from you within 10 business days, we will continue with our significance determination and enforcement decision. The final resolution of this matter will be conveyed in separate correspondence.

Since the NRC has not made a final determination in this matter, no Notice of Violation is being issued for this inspection finding at this time. In addition, please be advised that the number and characterization of the apparent violation described in the enclosed inspection report may change as a result of further NRC review.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's Agencywide Document Access and Management System (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

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However, the material enclosed herewith within contains Security-Related Information in accordance with 10 CFR 2.390(d)(1) and its disclosure to unauthorized individuals could present a security vulnerability. Therefore, the material in the enclosure will not be made available electronically for public inspection in the NRC Public Document Room ADAMS. If Security-Related Information is necessary to provide an acceptable response, please mark your entire response Security-Related Information in accordance with 10 CFR 2.390(d)(1) and follow the instructions for withholding in 10 CFR 2.390(b)(1). In accordance with 10 CFR 2.390(b)(1)(ii), the NRC is waiving the affidavit requirements for your response.

Sincerely,

/RA/

Jeffrey A. Clark, Acting Director
Division of Reactor Safety

Docket Nos: 50-313; 50-368
License Nos: DRP-51; NPF-6

Nonpublic Enclosure:

NRC Inspection Report 05000313/2013406; 05000368/2013406
w/Attachments:

1. Supplemental Information
2. Special Inspection Team Charter
3. Chronology for Safeguards Information Event

~~OFFICIAL USE ONLY - SECURITY RELATED INFORMATION~~

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