



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION IV  
611 RYAN PLAZA DRIVE, SUITE 400  
ARLINGTON, TEXAS 76011-4005

December 20, 2006

Jeffery S. Forbes, Vice President,  
Operations  
Arkansas Nuclear One  
Entergy Operations, Inc.  
1448 S.R. 333  
Russellville, Arkansas 72801-0967

SUBJECT: ARKANSAS NUCLEAR ONE, UNIT 2 - NRC PRESSURIZER REPLACEMENT  
INSPECTION REPORT 05000368/2006008

Dear Mr. Forbes:

On November 9, 2006, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at your Arkansas Nuclear One, Unit 2, facility. This inspection covered activities associated with the replacement of the pressurizer in Unit 2. No inspection of Unit 1 was performed under this report number. The enclosed report documents the inspection findings, which were discussed on November 9, 2006, with you and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection consisted of selected examination of procedures and representative records, observations of activities, and interviews with personnel.

Based on the results of this inspection, no findings of significance were identified.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

David L. Proulx, Chief  
Project Branch E  
Division of Reactor Projects

Docket: 50-368  
License: NPF-6

Entergy Operations, Inc.

-2-

Enclosure:

NRC Inspection Report 05000368/2006008

w/Attachments:

1. Supplemental Information
2. Security Related Information

cc w/o Attachment 2:

Senior Vice President  
& Chief Operating Officer  
Entergy Operations, Inc.  
P.O. Box 31995  
Jackson, MS 39286-1995

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Operations Support  
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Jackson, MS 39286-1995

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Electronic distribution by RIV:  
 Regional Administrator (**BSM1**)  
 DRP Director (**ATH**)  
 DRS Director (**DDC**)  
 DRS Deputy Director (**RJC1**)  
 Senior Resident Inspector (**RWD**)  
 Branch Chief, DRP/E (**DLP**)  
 Senior Project Engineer, DRP/E (**VGG**)  
 Team Leader, DRP/TSS (**MAS3**)  
 RITS Coordinator (**KEG**)  
 DRS STA (**DAP**)  
 D. Cullison, OEDO RIV Coordinator (**DGC**)  
**ROPreports**  
 ANO Site Secretary (**VLH**)

Report with Attachments

SUNSI Review Completed:   DLP   ADAMS:  Yes  No Initials:   DLP    
 Publicly Available  Non-Publicly Available  Sensitive  Non-Sensitive

Report without Attachments

SUNSI Review Completed:   DLP   ADAMS:  Yes  No Initials:   DLP    
 Publicly Available  Non-Publicly Available  Sensitive  Non-Sensitive

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SRI:DRP/E	SPE:DRP/E	C:DRS/OB	C:DRS/PSB
RWDeese	VGGaddy	RLNease	MPShannon
T - DLProulx	/RA/	/RA/	DAPowers for
12/18/06	12/6/06	12/7/06	12/14/06
RE:DRS/EB1	C:DRS/EB2	C:DRP/E	
JAClark	LJSmith	DLProulx	
/RA/	/RA/	/RA/	
12/11/06	12/15/06	12/20/06	

**U.S. NUCLEAR REGULATORY COMMISSION  
REGION IV**

Dockets: 50-368  
Licenses: NPF-6  
Report: 05000368/2006008  
Licensee: Entergy Operations, Inc.  
Facility: Arkansas Nuclear One, Unit 2  
Location: Junction of Hwy. 64W and Hwy. 333 South  
Russellville, Arkansas  
Dates: August 24 through November 9, 2006  
Inspectors: R. Deese, Senior Resident Inspector  
L. Carson II, Senior Health Physicist  
E. Crowe, Resident Inspector  
J. Kirkland, Project Engineer  
D. Livermore, Senior Project Engineer  
C. Paulk, Senior Reactor Inspector  
C. Young, Resident Inspector  
  
Accompanying Personnel: S. Makor, Reactor Inspector  
  
Approved By: David L. Proulx, Chief, Project Branch E  
Division of Reactor Projects

## SUMMARY OF FINDINGS

IR 05000368/2006008; 08/24/06 - 11/09/06; Arkansas Nuclear One, Unit 2; Integrated Resident and Regional Report of Pressurizer Replacement Activities.

This report covered a 3-month period of supplemental inspection by resident and regional inspectors. No findings were identified. The significance of most findings is indicated by their color (Green, White, Yellow, or Red) using Inspection Manual Chapter 0609, "Significance Determination Process." Findings for which the significance determination process does not apply may be Green or be assigned a severity level after NRC management's review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

A. NRC-Identified and Self-Revealing Findings

None.

B. Licensee-Identified Violations

None.

## REPORT DETAILS

### Summary of Plant Status

Unit 2 began the inspection period at 100 percent rated thermal power and remained there until September 19, 2006, when the unit was shut down for refueling and to replace the pressurizer. The unit was restarted on October 27, 2006, and achieved 66 percent rated thermal power on October 30, 2006. The unit was holding at 66 percent rated thermal power for physics testing when on October 30, 2006, a fire in a motor control center necessitated a plant shutdown. The unit was restarted on October 31, 2006, achieved 100 percent rated thermal power on November 1, 2006, and remained there for the remainder of the inspection period.

#### 1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems, Barrier Integrity

#### 1R02 Evaluation of Changes, Tests, or Experiments (71111.02)

##### a. Inspection Scope

The inspectors reviewed the effectiveness of the licensee's implementation of changes to the facility structures, systems, and components; risk-significant normal and emergency operating procedures; test programs; and the updated final safety analysis report in accordance with 10 CFR 50.59, "Changes, Tests, and Experiments." The inspectors reviewed the safety evaluations performed by the licensee dealing with the pressurizer replacement. The evaluations were reviewed to verify that licensee personnel had appropriately considered the conditions under which the licensee may make changes to the facility or procedures or conduct tests or experiments without prior NRC approval. Procedures, evaluations, screenings, and applicability determinations reviewed are listed in the attachment to this report. Additional observations will be documented in Section 1R02 of NRC Inspection Report 05000313/05000368-2006005.

##### b. Findings

No findings of significance were identified.

#### 1R08 Inservice Inspection Activities (71111.08)

##### a. Inspection Scope

The inspectors verified that observed examinations were performed in accordance with the specific site procedures and the applicable American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) requirements. During review of examinations, the inspectors verified that appropriate nondestructive examination procedures were used, examinations and conditions were as specified in the procedure, and test instrumentation or equipment was properly calibrated and within the allowable

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calibration period. The inspectors also verified the nondestructive examination certifications of the personnel who performed volumetric examinations. Finally, the inspectors observed that indications identified during the radiographic examinations were dispositioned in accordance with the ASME-qualified nondestructive examination procedures used to perform the examinations. Additional observations will be documented in Section 1R02 of NRC Inspection Report 05000313/05000368-2006005.

b. Findings

No findings of significance were identified.

1R13 Maintenance Risk Assessments and Emergent Work Control (71111.13)

a. Inspection Scope

Risk Assessment and Management of Risk

The inspectors reviewed the assessment activities listed below to verify: (1) performance of risk assessments when required by 10 CFR 50.65 (a)(4) and licensee procedures prior to changes in plant configuration for maintenance activities and plant operations; (2) the accuracy, adequacy, and completeness of the information considered in the risk assessment; (3) that the licensee recognizes, and/or enters as applicable, the appropriate licensee-established risk category according to the risk assessment results and licensee procedures; and (4) that the licensee identified and corrected problems related to maintenance risk assessments. Additional observations will be documented in Section 1R13 of NRC Inspection Report 05000313/05000368-2006005.

- August 24 through October 27, 2006, construction of the temporary rail system through the containment equipment hatch
- August 24 through October 27, 2006, reviewed procedures and plans for crane operation near common systems
- August 24 through October 27, 2006, evaluated controls and plans to minimize any adverse impact on Unit 1 and common systems
- August 24 through October 27, 2006, transport of the removed pressurizer into the containment building and transport of the replacement pressurizer offsite

b. Findings

No findings of significance were identified.

1R17 Permanent Plant Modifications (71111.17)

a. Inspection Scope

The inspectors reviewed key affected parameters associated with energy needs, materials/replacement components, timing, heat removal, control signals, equipment protection from hazards, operations, flowpaths, pressure boundary, ventilation boundary, structural, process medium properties, licensing basis, and failure modes for modifications associated with the replacement pressurizer. The inspectors verified that: (1) modification preparation, staging, and implementation does not impair emergency/abnormal operating procedure actions, key safety functions, or operator response to loss of key safety functions; (2) postmodification testing will maintain the plant in a safe configuration during testing by verifying that unintended system interactions will not occur, structures, systems, and components (SSCs) performance characteristics still meet the design basis, the appropriateness of modification design assumptions, and the modification test acceptance criteria has been met; and (3) the licensee has identified and implemented appropriate corrective actions associated with permanent plant modifications. Additional observations will be documented in Section 1R17 of NRC Inspection Report 05000313/05000368-2006005.

b. Findings

No findings of significance were identified.

1R20 Refueling and Outage Activities (71111.20)

a. Inspection Scope

The inspectors reviewed and observed selected portions of the following risk significant refueling items or outage activities to verify defense in depth commensurate with the outage risk control plan and compliance with the Technical Specifications: (1) the risk control plan; (2) tagging/clearance activities; (3) reactor coolant system (RCS) instrumentation; (4) electrical power; (5) decay heat removal; (6) spent fuel pool cooling; (7) inventory control; (8) reactivity control; (9) containment closure; (10) reduced inventory or midloop conditions; (11) refueling activities; (12) heatup and cooldown activities; and (13) licensee identification and implementation of appropriate corrective actions associated with refueling, outage, and replacement activities. Additional observations will be documented in Section 1R17 of NRC Inspection Report 05000313/05000368-2006005.

b. Findings

No findings of significance were identified.

1R23 Temporary Plant Modifications (71111.23)

a. Inspection Scope

For the temporary modifications listed below, the inspectors reviewed the Final Safety Analysis Report (FSAR), plant drawings, procedure requirements, and Technical Specifications to ensure that the four temporary modifications listed below were properly implemented. The inspectors: (1) verified that the modification did not have an affect on system operability/availability, (2) verified that the installation was consistent with the modification documents, (3) ensured that the postinstallation test results were satisfactory and that the impact of the temporary modification on permanently installed SSC's were supported by the test, (4) verified that the modifications were identified on control room drawings and that appropriate identification tags were placed on the affected drawings, and (5) verified that appropriate safety evaluations were completed. The inspectors verified the licensee identified and implemented any needed corrective actions associated with temporary modifications.

- September 19 - October 27, 2006, pressurizer removal and replacement
- September 19 - October 27, 2006, lifting and rigging preparations

b. Findings

No findings of significance were identified.

2. RADIATION SAFETY

Cornerstone: Occupational Radiation Safety

2OS1 Access Control To Radiologically Significant Areas (71121.01)

a. Inspection Scope

This area was inspected to assess the licensee's performance in implementing physical and administrative controls for airborne radioactivity areas, radiation areas, high radiation areas, and worker adherence to these controls with respect to the pressurizer replacement activities. The inspectors used the requirements in 10 CFR Part 20, the technical specifications, and the licensee's procedures required by technical specifications as criteria for determining compliance. During the inspection, the inspectors interviewed the radiation protection manager, radiation protection supervisors, and radiation workers. The inspectors performed independent radiation dose rate measurements and reviewed the following items:

- Controls (surveys, posting, and barricades) of radiation, high radiation, or airborne radioactivity areas
- Radiation work permits, procedures, engineering controls, and air sampler locations

- Conformity of electronic personal dosimeter alarm set points with survey indications and plant policy; workers' knowledge of required actions when their electronic personnel dosimeter noticeably malfunctions or alarms
- Barrier integrity and performance of engineering controls in airborne radioactivity areas
- Adequacy of the licensee's internal dose assessment for any actual internal exposure greater than 50 millirem Committed Effective Dose Equivalent
- Self-assessments related to the access control program since the last inspection; there were no audits, licensee event reports, and special reports documented
- Corrective action documents related to access controls
- Licensee actions in cases of repetitive deficiencies or significant individual deficiencies
- Radiation work permit briefings and worker instructions
- Adequacy of radiological controls such as, required surveys, radiation protection job coverage, and contamination controls during job performance
- Dosimetry placement in high radiation work areas with significant dose rate gradients
- Changes in licensee procedural controls of high dose rate - high radiation areas and very high radiation areas
- Posting and locking of entrances to all accessible high dose rate - high radiation areas and very high radiation areas
- Radiation worker and radiation protection technician performance with respect to radiation protection work requirements

The sample size for this inspection will be counted as part of NRC Inspection Report 05000313/05000368-2006005.

b. Findings

No findings of significance were identified.

2OS2 ALARA Planning and Controls (71121.02)

a. Inspection Scope

The inspectors assessed licensee performance with respect to maintaining individual and collective radiation exposures as low as is reasonably achievable (ALARA) during the pressurizer replacement. The inspectors used the requirements in 10 CFR Part 20 and

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the licensee's procedures required by technical specifications as criteria for determining compliance. The inspectors interviewed licensee personnel and reviewed:

- Pressurizer replacement work activities scheduled during the inspection period and associated work activity exposure estimates which were likely to result in the highest personnel collective exposures
- Site specific ALARA procedures
- ALARA work activity evaluations, exposure estimates, and exposure mitigation requirements
- Intended versus actual work activity doses and the reasons for any inconsistencies
- Interfaces between operations, radiation protection, maintenance, maintenance planning, scheduling and engineering groups
- Integration of ALARA requirements into work procedure and radiation work permit documents
- Person-hour estimates provided by maintenance planning and other groups to the radiation protection group with the actual work activity time requirements
- Shielding requests and dose/benefit analyses
- Dose rate reduction activities in work planning
- Method for adjusting exposure estimates, or re-planning work, when unexpected changes in scope or emergent work were encountered
- Exposure tracking system
- Use of engineering controls to achieve dose reductions and dose reduction benefits afforded by shielding
- Worker's use of the low dose waiting areas
- First-line job supervisors' contribution to ensuring work activities are conducted in a dose efficient manner
- Exposures of individuals from selected work groups
- Source-term control strategy or justifications for not pursuing such exposure reduction initiatives
- Radiation worker and radiation protection technician performance during work activities in radiation areas, airborne radioactivity areas, or high radiation areas

- Self-assessments related to the ALARA program since the last inspection; there were no audits and special reports documented

The sample size for this inspection will be counted as part of NRC Inspection Report 05000313/05000368-2006005.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA2 Identification and Resolution of Problems (71152)

Pressurizer Replacement Inspection (50003)

a. Inspection Scope

The inspectors reviewed the daily condition report summaries and nonconformance reports issued during the pressurizer replacement project for risk-significant issues to determine if the licensee was properly implementing the corrective action program. The inspectors verified that the licensee identified, evaluated, corrected, and trended in accordance with the program requirements. In addition, the inspectors reviewed the licensee's actions to identify and correct lessons learned from the Unit 1 steam generator and reactor vessel head replacement project and the Unit 2 steam generator replacement project.

b. Findings

No findings of significance were identified.

4OA5 Other Activities

Pressurizer Replacement Inspection (50003)

Design and Planning Inspections

a. Inspection Scope

The inspectors used the guidance in Inspection Procedure 50003, "Pressurizer Replacement Inspection," to perform the following pressurizer design and planning inspection activities:

Engineering and Technical Support

Inspections to review engineering and technical support activities were performed prior to and during the pressurizer replacement outage by resident and regional office-based specialist inspectors. The results of the inspection are documented in Sections 1R17 and 1R23. NRC Inspection Report 05000313/05000368-2006005, Sections 1R02,

1R08, and 1R17, will document additional activities that were performed as part of this review. This review verified that selected design changes and modifications to SSCs described in the FSAR were reviewed in accordance with 10 CFR 50.59. Additionally, key design aspects and modifications associated with the pressurizer replacement were also reviewed.

#### Lifting and Rigging

Inspections to review engineering design, modification, and analysis associated with pressurizer lifting and rigging activities were performed by resident and regional inspectors. This included: (1) crane and rigging equipment, (2) pressurizer component drop analysis, (3) safe load paths, and (4) load lay-down areas.

#### Radiation Protection

The review of radiation protection program controls, planning, and preparation in: (1) ALARA planning, (2) dose estimates and tracking, (3) exposure and contamination controls, (4) radioactive material management, (5) radiological work plans and controls, (6) emergency contingencies, and (7) project staffing and training plans is scheduled to be completed as part of the upcoming ALARA planning and controls inspection and documented in an upcoming integrated resident inspection report. Part of these activities were performed and are documented in Section 2OS1. Additional activities will be documented in NRC Inspection Report 05000313/05000368-2006005, Section 2OS1.

#### Additional Activities

Additional activities performed are documented in Attachment 2 which is designated and marked as "Security Related Information."

#### b. Findings

No findings of significance were identified.

#### Pressurizer Removal and Replacement Inspections

#### a. Inspection Scope

The inspectors used the guidance in Inspection Procedure 50003 to perform the following pressurizer removal and replacement inspection activities:

#### Welding and Nondestructive Examination Activities

Inspections were conducted to review welding and nondestructive examination activities including: (1) special procedures, (2) training and qualifications, (3) radiography results and work packages, and (4) completion of preservice NDE requirements for welds. This inspection was performed as part of Inspection Procedure 71111.08, "Inservice Inspection Activities." Additional observations will be documented in Section 1R08 of NRC Inspection Report 05000313/05000368-2006005.

### Lifting and Rigging

Inspections were conducted to review the preparations and procedures for rigging and heavy lifting including crane and rigging inspections, testing, equipment modifications, lay-down area preparations, and training for the following activities:

- Area preparation for the outside systems
- Pressurizer lifting device
- Upending device
- Old pressurizer removal
- New pressurizer placement
- Transport of old pressurizer offsite

### Major Structural Modifications

The inspectors observed the implementation and reviewed documentation related to structural modifications to facilitate pressurizer replacement, including the structural supports for the pressurizer, the temporary RCS piping structural supports, and all attached piping during all phases of removal and installation of the pressurizer.

### Outage Operating Conditions

The inspectors observed the establishment of operating conditions, including (1) defueling; (2) RCS draindown; (3) system isolation; (4) safety tagging; (5) radiation protection controls; (6) controls for excluding foreign material; and (7) installation, use, and removal of temporary services. Section 1R20 contains additional activities that were performed. NRC Inspection Reports 05000313/05000368-2006004 and 05000313/05000368-2006005, Section 1R20, will also documents activities performed during the pressurizer replacement outage.

### Storage of Removed Pressurizer

The inspectors observed the transport, storage, and radiological surveys of the removed pressurizer prior to shipment offsite. The radiological safety plans were also reviewed.

#### b. Findings

No findings of significance were identified.

### Postinstallation Verification and Testing Inspections

#### a. Inspection Scope

The inspectors used the guidance in Inspection Procedure 50003 to perform the following postinstallation verification and testing inspection activities. Selective inspections were performed on the following areas: (1) containment testing, (2) licensee's postinstallation inspections and verifications program and its implementation, (3) conduct of RCS leakage testing and review of test results,

(4) calibration and testing of instrumentation affected by pressurizer replacement, (5) procedures required equipment performance testing to confirm the design and to establish baseline measurements, (6) preservice inspection of new welds, and (7) pressurizer thermal and hydraulic performance.

b. Findings

No findings of significance were identified.

4OA6 Meetings, Including Exit

On October 6, 2006, the inspectors presented the Occupational Radiation Safety inspection results regarding the pressurizer replacement to Mr. T. Mitchell, General Manager, and other members of his staff who acknowledged the findings. The inspectors confirmed that proprietary information was not provided or examined during the inspection.

The engineering inspectors presented the results of the inservice inspection and welding review to Mr. J. Kowalewski, Director, Engineering, on October 10, 2006. Mr. Kowalewski acknowledged the inspection findings. The inspectors identified that they had not reviewed proprietary information

The inspectors presented the inspection results to Mr. T. Mitchell, General Manager, and other members of the licensee's management staff on November 9, 2006. The licensee acknowledged the findings presented. The inspectors noted that while proprietary information was reviewed, none would be included in this report.

ATTACHMENT: SUPPLEMENTAL INFORMATION AND SECURITY RELATED INFORMATION

## **SUPPLEMENTAL INFORMATION**

### **KEY POINTS OF CONTACT**

#### Licensee Personnel

J. Bacquet, ALARA Supervisor  
R. Barnes, Manager, Planning, Scheduling, and Outages  
B. Berryman, Manager, Operations Unit 1  
J. Browning, Manager, Operations Unit 2  
S. Cotton, Manager, Training  
B. Daiber, Supervisor, Systems Engineering  
F. Daubenmeyer, Project Manager  
J. Eichenberger, Manager, Corrective Actions and Assessments  
J. Forbes, Vice President, Operations  
J. Giles, Manager, Technical Support  
M. Ginsberg, Supervisor, Engineering Programs and Components  
A. Hawkins, Licensing Specialist  
J. Hoffpauir, Manager, Maintenance  
R. Holeyfield, Manager, Emergency Planning  
M. Huff, Supervisor, Project Engineering  
D. James, Manager, Licensing  
W. James, Manager, Engineering Projects  
J. Kowalewski, Director, Engineering  
T. Marlow, Director, Nuclear Safety Assurance  
J. Miller, Jr., Manager, System Engineering  
T. Mitchell, General Manager, Plant Operations  
D. Moore, Manager, Radiation Protection  
C. Reasoner, Manager, Engineering Programs and Components  
M. Ruder, Corrective Actions and Assessments  
D. Stoltz, ALARA Coordinator  
C. Tyrone, Manager, Quality Assurance  
F. Van Buskirk, Licensing Specialist  
P. Williams, Supervisor, Systems Engineering  
M. Woodby, Manager, Design Engineering

### **LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED**

#### Opened and Closed

None

#### Closed

None

#### Discussed

None

## LIST OF DOCUMENTS REVIEWED

In addition to the documents referred to in the inspection report, the following documents were selected and reviewed by the inspectors to accomplish the objectives and scope of the inspection and to support any findings:

### Calculations

90-E-0116-01	93-E-0066-01	99-E-0055-01	CALC-1.3.7.4.1
91-D-2016-02	94-E-0069-01	A2-SA2005-001	CALC-2.7.5.4.1
91-D-2016-10	99-E-0051-01		

### Condition Reports

ANO-2-2006-0119	ANO-2-2006-1178	ANO-2-2006-1641	ANO-2-2006-1966
ANO-2-2006-0143	ANO-2-2006-1207	ANO-2-2006-1652	ANO-2-2006-2045
ANO-2-2006-0202	ANO-2-2006-1250	ANO-2-2006-1657	ANO-2-2006-2048
ANO-2-2006-0231	ANO-2-2006-1257	ANO-2-2006-1661	ANO-2-2006-2157
ANO-2-2006-0249	ANO-2-2006-1265	ANO-2-2006-1679	ANO-2-2006-2173
ANO-2-2006-0264	ANO-2-2006-1319	ANO-2-2006-1720	ANO-2-2006-2174
ANO-2-2006-0305	ANO-2-2006-1361	ANO-2-2006-1749	ANO-2-2006-2223
ANO-2-2006-0306	ANO-2-2006-1461	ANO-2-2006-1765	ANO-2-2006-2230
ANO-2-2006-0315	ANO-2-2006-1480	ANO-2-2006-1799	ANO-2-2006-2237
ANO-2-2006-0342	ANO-2-2006-1482	ANO-2-2006-1818	ANO-2-2006-2257
ANO-2-2006-0467	ANO-2-2006-1517	ANO-2-2006-1835	ANO-2-2006-2267
ANO-2-2006-0686	ANO-2-2006-1525	ANO-2-2006-1858	ANO-2-2006-2273
ANO-2-2006-0806	ANO-2-2006-1608	ANO-2-2006-1891	ANO-2-2006-2311
ANO-2-2006-0901	ANO-2-2006-1609	ANO-2-2006-1896	ANO-2-2006-2314
ANO-2-2006-1033	ANO-2-2006-1618	ANO-2-2006-1906	ANO-2-2006-2315
ANO-2-2006-1086	ANO-2-2006-1631	ANO-2-2006-1919	ANO-2-2006-2333
ANO-2-2006-1165	ANO-2-2006-1632	ANO-2-2006-1930	ANO-2-2006-2347
ANO-2-2006-1177	ANO-2-2006-1633	ANO-2-2006-1956	ANO-2-2006-2358

Engineering Requests

ER-ANO-2002-0836-004  
ER-ANO-2003-0245-020  
ER-ANO-2002-0836-007  
ER-ANO-2002-0836-002  
ER-ANO-2002-0836-003  
ER-ANO-2002-0836-005  
ER-ANO-2002-0836-006  
ER-ANO-2002-0836-012

Miscellaneous

LER 87-003-00, "Pressurizer Heater Rupture Results in Reactor Coolant Boundary Leakage"  
Catalog ID 0032034759  
Catalog ID 0032034609  
Contract Requisition 1339997  
Design Change Package 80-2155  
Design Change Package 91-2016  
ERT-A2-2002-0836-01-00, Pressurizer Heater Testing  
Operating Experience Report 23294  
Purchase Order 10130928  
Material Requisition 01273405

Operating Procedures

Number	Title	Revision
EN-LI-101	10 CFR 50.59 Review Program	7
EN-LI-102	Corrective Action Program	3
EN-LI-120	Safety Review Committee	0
EN-IS-102	Confined Space Program	2

EN-MA-101	Control of Maintenance	2
EN-OM-119	On-Site Safety Review Committee	0
ENS-DC-105	Configuration Management	2
ENS-DC-112	Engineering Request and Project Initiation Process	4
ENS-DC-115	ER Response Development	6
EN-EV-112	Chemical Control Program	1
1015.036	Containment Building Closeout	11
1022.013	Preparation and Conduct of Refueling Activities	6
2203.008	Natural Emergencies	9
2304.200	Unit 2 Pressurizer Pressure Calibration	10
2305.006	Cold Shutdown Valve Testing	17
5000.009	Repair/Replacement Program Administration	4
5120.110	Review and Approval of Contractor Welding Programs	3

Drawings:

924664-1	C-2177	E-2080
924664-2	C-2183	—2001
C-2026A	D-AB-22141-M01	—2136
C-2035	DSW-RPZR-06-09	Westinghouse 10010E81
C-2176		

Work Requests:

84639  
85454  
85839  
85888

Work Orders:

00069224  
00082807  
00082810

00082811

00084635

00084634

Section 2OS2: Access Controls to Radiologically Significant Areas (71121.01)

Section 2OS2: ALARA Planning and Controls (71121.02)

Condition Reports:

ANO-2-2006-1575

ANO-2-2006-1679

ANO-2-2006-1766

ANO-2-2006-1608

ANO-2-2006-1765

Self-Assessments

Self-Assessment Report, QS-2006-ANO-007, 2R18 Radiation Protection Outage Planning Review

Radiation Work Permits

RWP 2006-2501, Support Activities for Pressurizer Replacement

RWP 2006-2502, Remove and Replace Pressurizer

Procedures

EN-RP-104

Personnel Contamination Events

Revision 3

EN-RP-108

Radiation Protection Posting

Revision 3

EN-RP-131

Air Sampling

Revision 1

EN-RP-203

Dose Assessment

Revision 0

PL-182

Radiation Protection Expectations and Standards

Revision 1

Miscellaneous Documents

Alpha Monitoring Plan, Revised August 22, 2006

## LIST OF ACRONYMS

ALARA	as low as reasonably achievable
ANO	Arkansas Nuclear One
ASME	American Society of Mechanical Engineers
CFR	<i>Code of Federal Regulations</i>
FSAR	Final Safety Analysis Report
RCS	reactor coolant system
SSC	structures, systems, and components